

RADIATION PHENOMENA AT EDGE AND DIVERTOR

U. Samm

Institut für Plasmaphysik

Forschungszentrum Jülich, Association EURATOM-KFA, Jülich, Germany, Member in the Trilateral Euregio Cluster

ABSTRACT

The concept of a cold radiative plasma boundary is presented as a possible solution of the energy exhaust problem in a fusion reactor. The most relevant processes which determine level and location of the radiation from low-Z impurities are discussed in detail. Experimental results are used to demonstrate the general feasibility of generating a stable and quasi-stationary plasma with a cold radiating layer on a high power level. Furthermore, the limitations of the concept are briefly analyzed addressing feed-back control of the impurity level, thermal stability, particle transport and impurity contamination.

I. THE CONCEPT OF THE "COLD RADIATIVE PLASMA BOUNDARY"

One of the outstanding challenges on the way to a fusion reactor is the problem of energy exhaust¹. The α -particle heating will generate a radial power flow of about 0.3 MW/m^2 which ultimately has to be exhausted via the wall structures (ITER: $P_\alpha \approx 100 \text{ MW}$). If most of this power would enter the scrape-off-layer (SOL) convectively the power flowing parallel to the fieldlines would be in the order of 500 MW/m^2 . Even with optimistic assumptions about possible means for reducing the power load density on the wall surface, e.g. by an inclined target plate, the peak load on the first wall will come close to 20 MW/m^2 - a value too high for stationary operation. The necessary reduction of the power load can be achieved by distributing a substantial fraction of the power on other parts of the wall. A promising means to achieve this is given by line radiation from impurities, which allows to distribute the power load on large areas (the whole vessel/divertor wall). In case this line radiation would be confined to the plasma boundary, cooling the plasma edge and leaving the plasma core essentially unaffected, we have a situation as considered in the concept of the "*cold radiative plasma boundary*".

I.A. The relevant processes and the radiation characteristics

Impurities enter the plasma in general as neutral particles - as atoms or molecules. Due to electron impact the particles are dissociated and ionized. The characteristic penetration depth λ_i of the particles is given by their ionization time

$$\tau_i = 1/(n_e \langle \sigma_i v \rangle) \quad (1)$$

and the velocity of the atoms v_A according to

$$\lambda_i = v_A \tau_i, \quad (2)$$

where n_e is the local electron density and $\langle \sigma_i v \rangle$ is the corresponding rate coefficient for ionization. Typical values of λ_i for neutrals are in the order of 1 cm ; e.g. for neon injected via a gas puff system or for sputtered silicon. After ionization the charged particles are transported in radial direction by cross-field diffusion. The characteristic transport length is given again by equ. 2, but with v_A replaced but an effective radial diffusion velocity v_\perp . Step by step ionization to higher states occurs as the particles penetrate into regions with higher electron temperatures T_e . Recombination does not play a role at the plasma edge since the typical times for recombination at densities in the range of $n_e = 10^{13} \text{ cm}^{-3}$ are for most cases orders of magnitude longer than the average dwell time of the particles in the plasma. In contrast, inside a divertor with densities sometimes above 10^{14} cm^{-3} recombination may become relevant. However, in an edge plasma the distribution of the ion charge states forms radial nested regions with increasing charge towards the plasma centre.

Excitation of the particles is closely linked to the ionisation process. The excitation rate is given by

$$\phi_z^{ij} = n_e n_{iz} \langle \sigma_e v \rangle_z^{ij}, \quad (3)$$

where n_{iz} is the local ion density of a given charge state z and $\langle \sigma_e v \rangle_z^{ij}$ is the rate coefficient for the transition $i \rightarrow j$. Excitation can only take place during the life time τ_i of the ion, thus the total number of excitations is $N_z^{ij} = \phi_z^{ij} \tau_i$ or the number of *excitations per ion* is given by the ratio of the rate coefficients

$$\alpha_z^{ij} = \langle \sigma_e v \rangle_z^{ij} / \langle \sigma_i v \rangle \quad (4)$$

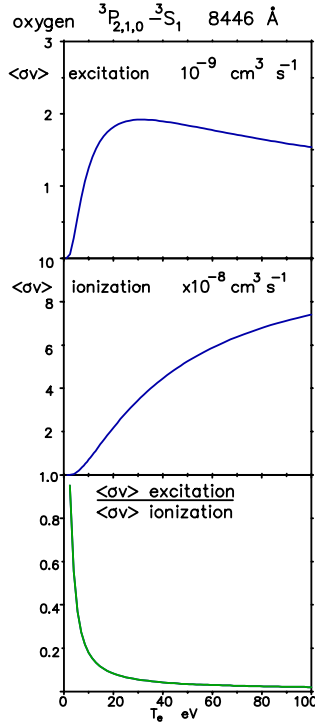


Figure 1 Example for the T_e -dependence of rate coefficients and their ratio

The rate coefficients depend mainly on T_e . A typical example is shown in Fig. 1 for a certain transition with oxygen^{2,3}. By multiplying α_z^{ij} with the excitation energy ΔE^j one obtains the power which can be radiated from this state. Summation over all energy levels and charge states with relative concentrations c_z of the impurity particle gives the total energy radiated during its life time in the plasma.

$$E_{\text{rad}} = \sum_z c_z \sum_{ij} (\alpha_z^{ij} \Delta E^j) \quad (5)$$

The term E_{rad} is called the *radiation potential*. The total power radiated P_{rad} is then obtained by multiplying E_{rad} with the total flux Γ_i of the impurity particles entering the plasma (usually as neutrals)

$$P_{\text{rad}} = E_{\text{rad}} \Gamma_i \quad (6)$$

Radial transport and the T_e profiles determine the relative population of the various ionization stages. The ionization time τ_i for certain ionization states depends on the local T_e values. However, radial transport limits essentially the overall life time (dwell time) of the specific ion in the plasma: either by neutralization at the wall or by ionization to higher states.

If transport is diminishing ($v_{\perp} \rightarrow 0$) we have the special case of *coronal equilibrium*, where the dwell time approaches infinity and

radiative recombination becomes relevant. In this case the radiation potential has no meaning anymore. Instead, the *radiation function* $P(T_e)$ can be used to calculate the local radiation density p_{rad} :

$$p_{\text{rad}} = P(T_e) n_e n_i \quad (7)$$

Here n_i is the ion density summed over all ionization states. The total power radiated is obtained by integrating over the whole plasma volume

$$P_{\text{rad}} = \iiint n_e n_i P(T_e) dx dy dz \quad (8)$$

An example for $P(T_e)$ of some elements^{4,5} is shown in Fig. 2.

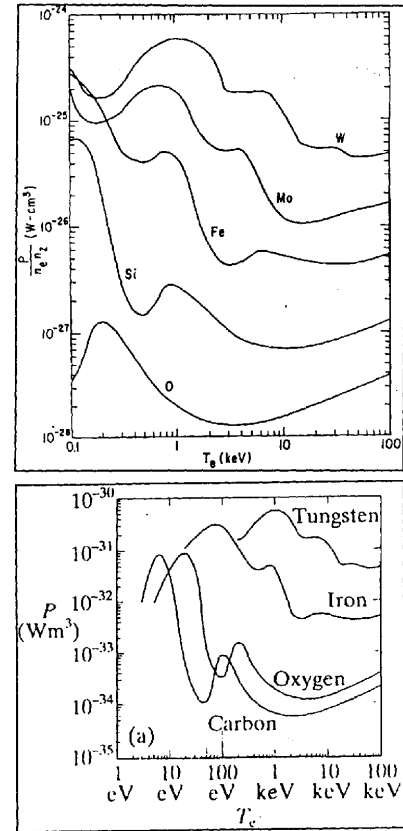


Figure 2 Radiation function for coronal equilibrium from Jensen, Post⁴ and Summers, McWhirter⁵

The advantage of relation 6 is that Γ_i can be measured much easier than the radial distribution of ion densities as required in equ. 8. The radiation potential helps to identify the impurity flux needed for a certain power level of radiation and to qualify the capability for radiation cooling of different impurities, whereas equ. 8 relates radiation to the impurity density at the boundary. Both terms, E_{rad} and $P(T_e)$, depend on transport properties of the impurity particles and are most sensitive to the local electron temperature.

But coronal equilibrium is not valid at the plasma boundary since it requires a balance between ionization and radiative recombination. Instead, the ionization equilibrium is determined by transport. In 0-d calculations ($v=0$, constant T_e) the ionization equilibrium may be

approximated by introducing a finite lifetime⁶. But a more accurate calculation requires at least a 1-d treatment of the transport processes. However, both methods show that the existence of lower ionization states can be shifted significantly towards higher temperatures yielding much more radiation at higher T_e compared to the coronal equilibrium case. For example the peak radiation of carbon shown in Fig. 3, calculated with a 1-d transport model⁷, occurs at $T_e=35$ eV, a value much higher than the peak appearing at $T_e=7-8$ eV in Fig. 2. We conclude that the transport dominated ionization equilibrium is advantageous for providing a high radiation level for low-Z elements.

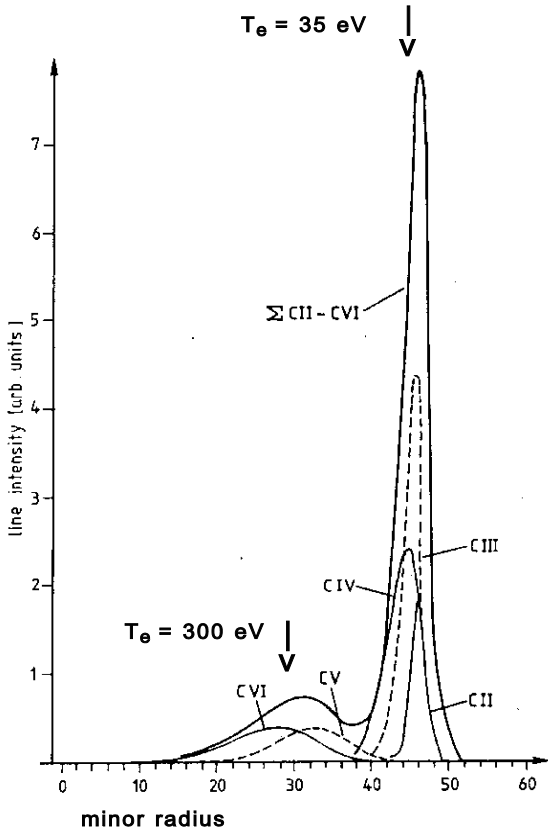


Figure 3 Radial distribution of the radiated power from carbon ions calculated with a 1d-transport code for TEXTOR conditions

In TEXTOR experiments E_{rad} has been determined from measurements of impurity fluxes Γ_i and P_{rad} ⁸ and has been calculated with a 1-d transport code⁹. Results for the impurities C, O and Ne are shown in Fig. 4, where E_{rad} is plotted versus the T_e values at which the particles enter the plasma. When comparing E_{rad} and $P(T_e)$ we have to keep in mind that in a 0-d calculation (e.g. Fig. 1) all the radiation is emitted at a specific T_e value, whereas the T_e values shown for the 1-d case (Fig. 4) represent the starting points of radial profiles.

I.B. T_e - dependence and extent of the radiating boundary

The characteristic dependence of radiation per particle on T_e is already evident from the behaviour of a single transition as shown in Fig. 1, where the ratio of rate coefficients exhibits a pronounced increase with decreasing T_e . This is typical for all transitions and charge states. Thus, after summation also E_{rad} as well as $P(T_e)$ will have a similar T_e -dependence as can be seen in Figs. 2 and 4. Such a behaviour is favorable for generating a cold radiating boundary, since with a decrease of T_e the radiation level is rising without employing more impurities. On the other hand, this property can lead to instabilities as will be discussed later.

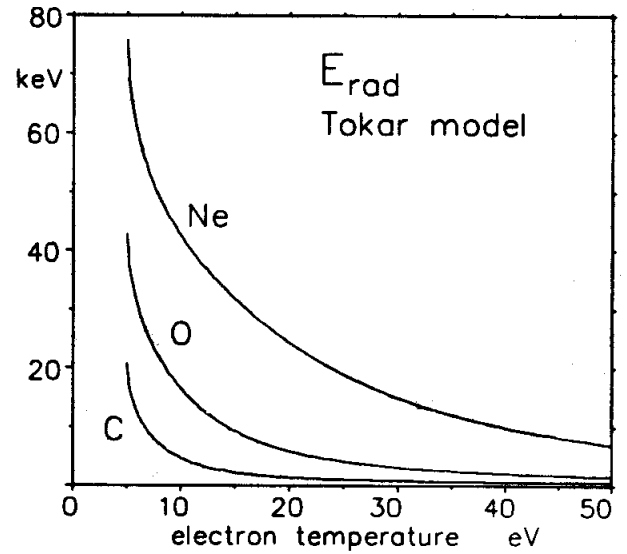


Figure 4 The radiation potential as a function of the edge electron temperature for TEXTOR conditions

The number of excitations per particle can also be illustrated with the following formulas for the Born-Coulomb rate coefficients for ionization and excitation³

$$\langle \sigma_i v \rangle = 10^{-8} \left| \frac{R_y}{E_0} \right|^{3/2} \exp\left(-\frac{E_0}{T_e}\right) Q_i \frac{G_i(E_0/T_e)}{2l+1} \quad (9)$$

$$\langle \sigma_e v \rangle \geq 10^{-8} \left| \frac{R_y}{E_0 - E_1} \right|^{3/2} \left(\frac{E_1}{E_0} \right)^{3/2} \exp\left(-\frac{\Delta E_0}{T_e}\right) Q_k \frac{G(\Delta E_0/T_e)}{2l+1}$$

where the following parameters are used : the Rydberg constant R_y , the ionization energy from the ground state E_0 , the ionization energy from the excited state E_1 and $\Delta E = E_0 - E_1$. Q_i is a parameter depending on the quantum numbers of the electron shell. If we neglect the weak variation of the factors G and G_i for a rough estimate and multiply the number of excitations per ion by the average excitation energy ΔE then we obtain the following relation for the power radiated from this level

$$\Delta E \frac{\langle \sigma_e v \rangle}{\langle \sigma_i v \rangle} \propto \exp(E_1/T_e) \frac{E_1^{3/2}}{\Delta E^{1/2}} \quad (11)$$

which clearly exhibits the strong drop of radiation with increasing T_e .

In the case of an edge plasma, where the plasma parameters vary strongly in radial direction, the *location of radiation* is essentially determined by the actual T_e -profile. According to the results obtained from a 1-d transport code shown in Fig. 3 for the case of carbon the different ionization states contribute in a distinct way. The Li- and Be-like states (C III, C IV) contribute more (factor of 2-3) to the total radiation than the He- or H-like ions (C V, C VI). This is due to the fact that the relative difference between ionization and excitation energies is significantly larger in the case of the Li- and Be-like ions leading to more excitations per ionization. The influence of these energy levels is also reflected in eqs. 9-11.

electr. config. like	element								
	Li	Be	B	C	N	O	F	Ne	Si
	3	4	5	6	7	8	9	10	14
H	122	218	340	490	667	871	1103	1360	2673
He	75	153	259	392	552	739	953	1196	2437
Li	5	18	38	65	98	138	185	239	523
Be		9	25	48	76	114	157	208	476
B			8	24	48	77	114	158	401
C				11	30	55	87	126	351
N					14	35	63	92	303
O						14	35	63	246
F							17	41	205
Ne								22	166
Na									54
Mg									33
Al									16
Si									8
ΣE_i all	202	398	670	1030	1485	2043	2714	3505	7893
ΣE_i -Li	5	27	71	148	267	433	658	949	2783

Table 1 Ionization energies in eV and their sum over all levels (ΣE_i all) and excluding the H- and He-like states (ΣE_i -Li)

Most important is the fact that the CVI and CV radiation is distributed over a broader radial range than the radiation from the lower ionization states as is evident from Fig. 3. As a consequence the peak radiation densities differ by about a factor of 10. This is beneficial for the purpose of generating a radiating plasma boundary. The larger spread of radiation for the H- and He-like ions is caused by their relative high ionization energies E_0 leading to an enhancement of their ionization length (equ.2). The jump of E_0 when going from the Li-like to the He-like state is typical for all

elements as can be seen in table 1.

Based on these properties we may conclude that the extent of the radiating layer towards higher T_e is limited essentially by the ionization of the Li-like ions. A crude estimate about the T_e values at which a particle is ionized can be obtained by calculating the ionization length λ_i according to equ. 2. The strong T_e -dependence of $\langle \sigma_i v \rangle$ will cause a drastic drop of λ_i with increasing temperature. We may define the location at which λ_i drops below a certain value as the radius of ionization (and radiation). Experiments as well as model calculations show that the ionization occurs at an electron temperature corresponding to 1/2 to 1/3 of the ionization energy E_0 of the particular ion. Therefore a guess at the extent of a radiating layer is given by the location of $T_e = E_0/2$ for the Li-like ion stage. It is interesting to compare the radiated energy per particle E_{rad} with the total energy needed for ionization. Comparing table 1 with figure 4 shows that about 10-40 times more energy can be radiated than is necessary for ionization up to the He-like state (ΣE_i (-Li)).

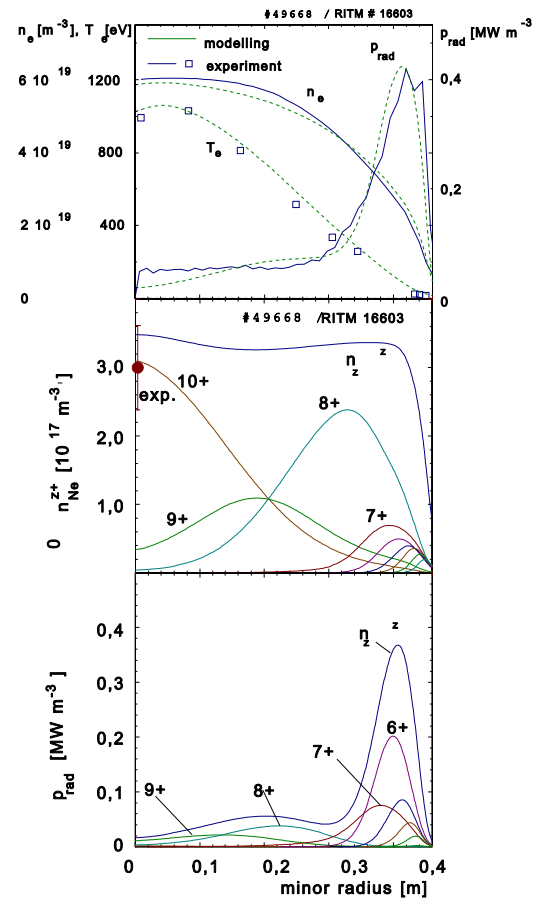


Figure 5 Radiation cooling with neon; profiles of n_e , T_e and radiated power measured in TEXTOR and model calculations with the code RITM⁹ showing the neon ion densities and the corresponding radiation

For neon the main radiation is expected to go up to $T_e \approx 120$ eV. This can be confirmed by experimental data obtained on TEXTOR shown in Fig. 5. With neon puffing a high level of radiation ($P_{rad}/P_{heat}=85\%$ in this case) has been obtained. T_e in the centre

remains constant (or even increases slightly), whereas T_e at the edge is significantly reduced. This proves that selective edge cooling is possible. The drop in T_e starts at a radius of $r/a \approx 0.75$, which coincides with the extent of the main part of the radiative layer and an electron temperature of about 120 eV. A strong drop of the radiation occurs when the Li-like ions (Ne^{7+}) are ionized, as is clearly seen in Fig. 5. According to this simple rule we may estimate the extent of the radiative layer for all tokamaks only based on the specific T_e -profile. This is the main effect. Variation of transport will be a second order effect. An important consequence is that in future devices like ITER, where much higher T_e values will develop, the radiative layer will be confined to a much smaller fraction of the whole volume.

I.C. Radiation in a divertor configuration

The strong plasma flow inside the divertor towards the divertor plates changes the radiation properties of the edge plasma significantly^{10,11,12}. Neutral particles recycling at the target plate are ionized inside the divertor leading to rather high electron densities in the divertor chamber. The impurities released from the target plates, which are also mostly ionized inside the divertor, are partly flushed back due to friction forces caused by the strong flow of the background plasma, in particular in regions close to the target plate, or are pushed upstream due to thermal forces caused by the temperature gradient along the magnetic field lines. Thus, on the one hand we get radiation from impurities inside the divertor and on the other hand impurities reaching the main chamber can generate a radiating belt at the edge plasma in a similar way as for a limiter configuration. Although the radiation density inside the divertor is generally much higher than that of the radiating belt in the main chamber, the total radiated power can be about the same because of the much larger volume in the main chamber.

III. EXPERIMENTAL RESULTS AND LIMITATIONS OF THE CONCEPT

III.A. The radiation level

On TEXTOR it has been shown first that already with light species, like Ne or Si, which provide selective edge cooling, a significant fraction of the total power can be radiated^{13,14}. These light impurities may serve for the realization of the concept of the *cold radiating plasma boundary*. The discharges are stable and quasi-stationary. In fig.6 some data from a series of shots is shown with various amounts of neon at a medium line-averaged electron density ($n_e=4 \cdot 10^{13} \text{cm}^{-3}$), a medium heating power level (1.6 MW) and a flat top time for the radiation of about 1s. The radiation level can rise close to 100%. Similar results have been obtained at a high power level, where a radial power flux density of 0.11 MW/m² to the edge is obtained. In this case the convective transport entering the scrape-off layer can be reduced by radiation to 0.02 MW/m². In TEXTOR it has been demonstrated that such conditions can be kept quasi-stationary¹⁵ (see also fig.8).

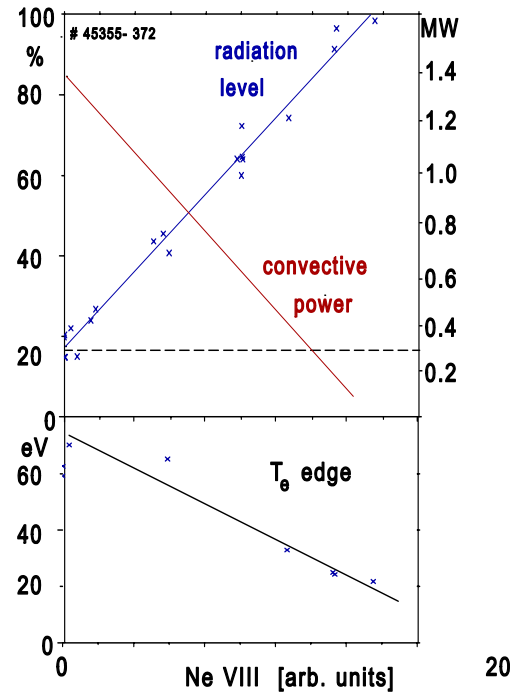


Figure 6 Radiation level, convective power to limiter and edge T_e as a function of the amount of injected neon

Also in tokamaks with a *divertor* a high fraction of the heating power can be removed by radiation from the plasma boundary. This has been demonstrated first on ASDEX-Upgrade with the help of neon injection^{16,17}. An essential element of both experiments - TEXTOR and ASDEX-U – is the use of a feedback system for neon as a key element for achieving quasi-stationary conditions. Meanwhile radiation cooling experiments have been performed on many tokamaks by injecting Ne, Ar and N. The critical issues in these experiments with a radiative plasma boundary as feed-back control, thermal stability, energy confinement and impurity contamination are discussed in the following.

III.B. Special properties, limitations and critical issues of the concept

Feed-back control of the impurity level

Quasi-stationary operation of discharges at a radiation level close to 100% is only possible with a very well controlled impurity level. At such a high radiation level already a tiny increase of the impurity concentration can lead to a major disruption. It turns out that a feedback system to control the impurity level is indispensable. In case of neon this has been realized with the help of a gas-puff system and by employing as a sink either a pump limiter (ALT-II in TEXTOR) or a pumped divertor (ASDEX-U). For the control variable we may use a NeVIII line (TEXTOR), which represents the total radiation from neon sufficiently well, or directly a fast bolometer signal giving the total radiation (ASDEX-U, JET).

The control of silicon is more difficult. In a siliconized machine the intrinsic source for Si is sputtering, a process which depends strongly on the plasma edge temperature. Generally, with increasing electron density the silicon radiation decreases due to a drop of the sputtering yield. Additional radiation from Si can be obtained e.g. by Si_2H_6 puffing, but the possible variation is limited due to the unavoidable density increase arising from the injection of molecules containing hydrogen. The maximum radiation level obtained so far was about $\gamma=75\%$ with full di-silane fuelling¹⁸.

Thermal stability

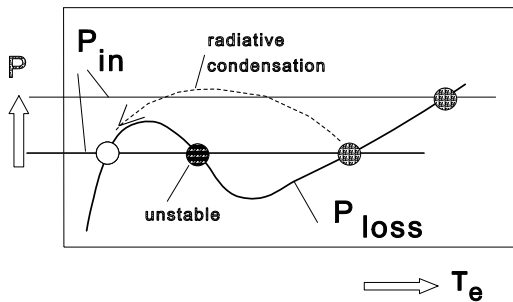


Figure 7 Stable and unstable regimes in the local heat flux balance of a radiating edge plasma

Fig. 7 shows schematically a typical T_e -dependence of the local energy losses P_{loss} in an edge plasma. In steady state the local power input (heating, conduction, convection) has to balance the losses. At high electron temperatures the losses are dominated by charge exchange and ionization processes and therefore increase with T_e , whereas in a cold plasma the radiative losses dominate showing an increase with decreasing T_e (compare with Fig. 4). At the low end of the T_e range the losses decrease again because the processes leading to radiation diminish (e.g. particle sources). At high relative input power the solution is in a range where $\delta P_{\text{loss}}/\delta T_e > 0$, i.e. the system is stable. When lowering the power level (equivalent is increasing the radiation level) a second stable solution at low T_e may be possible, as is indicated in Fig. 7. An abrupt transition to this state is often observed. On a time scale of 10th of ms zones of cold plasma develop. In low and medium power discharges this can occur poloidally symmetric. Then the whole hot plasma column appears to shrink radially, which is called *detachment* because the plasma flow to the limiters is nearly vanishing¹⁹. At high power levels, where normally strong in/out poloidal asymmetries in the heat transport are present, we observe a different phenomenon. Instead of a full detachment a toroidal belt of a cold, high density plasma develops at the location of lowest radial heat flux, that is at the high field side of the tokamak. This highly radiating belt is called a "MARFE" (Multifaceted Asymmetric Radiation From the Edge)²⁰. In such a MARFE the electron density is much higher and the electron temperature much lower ($n_e > 10^{14} \text{ cm}^{-3}$ $T_e < 5 \text{ eV}$) than in neighbouring plasma regions²¹. The total power radiated from a MARFE is in the order of 20%-50% of the total input power. It is important to note that this radiation may lead in the vicinity of the MARFE to significant peak loads on the wall. Experimentally,

MARFEs can be avoided by operating not too close to the density limit.

The radiation inside the divertor can lead to the phenomenon of *divertor detachment*. In contrast to the detachment with edge radiation, where the cold plasma layer moves in radial direction (radial temperature gradient), the peak radiation inside the divertor moves parallel to the field lines (parallel temperature gradient) away from the target plate. As a consequence the plasma flow to the plates diminishes. At the x-point normally the heat flux density is high enough to stop this development resulting in a strongly radiating zone located around the x-point which is called x-point MARFE²². It is important to avoid too strong a detachment, because the divertor function would be lost in case of an extremely cold divertor plasma where the ionization front moves too far away from the target.

Energy confinement

The concept of a cold radiative plasma boundary can only be applied successfully to a fusion reactor if it is compatible with the important requirements on confinement. The energy confinement must be kept sufficiently high and the particle confinement, in particular the confinement of He ash, must be low enough to allow for a stationary burning plasma.

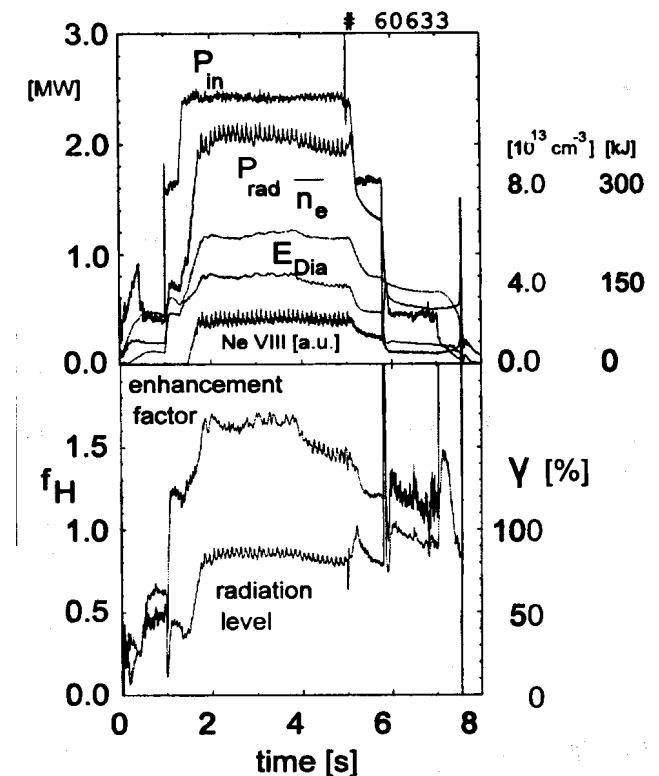


Figure 8 Improved confinement in a long pulse discharge with edge cooling by neon

It is remarkable that under special circumstances strong radiative edge cooling can increase the global energy confinement time. This

phenomenon has already been observed in earlier experiments²³, but on a high power level and in conjunction with controlled edge cooling it has first been demonstrated on TEXTOR and ASDEX-U. In fig. 8 a TEXTOR discharge is shown where a clear transition to increased energy confinement is observed with neon injection (displayed by the time trace of the Ne-VIII line radiation). About 80% of the total input power is radiated. The quality of the energy confinement corresponds to ELM-free H-mode confinement in divertor tokamaks. In TEXTOR this type of discharge is called *Radiative Improved mode*²⁴. A set of ASDEX-U discharges with different radiation levels is shown in fig.9. It is demonstrated that under certain conditions (amount of neon) the energy confinement time follows the H-mode scaling (high confinement mode). Since the power onto the divertor plate nearly diminishes this is called the Complete Detached H-mode (CDH-mode)^{16,17}. It is remarkable that also under L-conditions without the characteristic H-mode signatures (e.g. edge transport barrier and ELMs) - in a discharge scenario very similar to the RI-mode- a significant improvement of confinement is observed coming close to that of the CDH-mode (circles in fig 9 at high P_{rad}/P_{loss}).

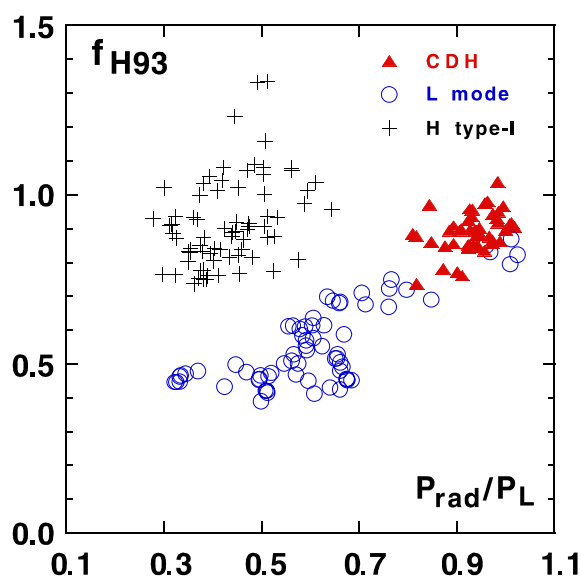


Figure 9 energy confinement normalized to the H-mode scaling for different radiation level with (CDH) and without (L-, H-mode) neon injection in ASDEX-Upgrade

The improvement of energy confinement in radiating mantle plasmas is attributed to the suppression of the so called *toroidal ion temperature gradient driven mode* as a consequence of the impurity injection²⁵. To extrapolate this very attractive confinement regime to reactor scale plasmas experiments in large devices like JET are being conducted with the aim to combine high density operation and good confinement with power exhaust by radiation²⁶. In this context the influence of a radiating edge to H-mode properties is a crucial issue. It has been observed that above a certain radiation level the edge plasma pressure gradients can be reduced such that the H-mode properties are lost with the consequence of reduced confinement. It is an issue of ongoing research to optimize the combination of high confinement and the feed-back injection of

radiating impurities.

Radial transport and He-exhaust

We can identify two aspects in which impurity transport is a critical issue with respect to radiation cooling: a) efficient *He removal* (ash exhaust) has to be possible with a "cold" plasma edge and b) the build up of *impurity concentrations in the centre* - helium as well as other impurities including those used for radiation cooling - must stay below a critical level.

For efficient He-removal extreme divertor detachment has to be avoided, because the pumping efficiency might become too small. A much more critical issue is the amount of impurities being transported and accumulated in the plasma centre. From the conditions for a burning fusion plasma follows an upper limit for the impurity concentration in the plasma centre²⁷. The radial transport of helium and its exhaust properties determine the margin left for other impurities than helium acceptable in the plasma core. Taking into account also the impurities coming from erosion processes a certain margin will be left for those special impurities needed to enhance and control the radiation cooling. From this follows also a maximum level of radiation. Thus the concept of a radiating boundary is only realistic under conditions with sufficient He-exhaust and well controlled impurity contamination from wall erosion.

IV. CONCLUSIONS

Edge radiation can significantly help to solve the power exhaust problem: radiation distributes the energy on large areas of the wall, thus avoiding or reducing the peak power loads which appear at the strike point of the divertor target. The concept of a radiating plasma mantle is now also part of the plasma scenarios foreseen for ITER. In ITER a significant level of radiation is foreseen (at least 60 % of the effective heating power) to bring the peak load to the wetted area down to values below 10 MW/m². With a maximum of radiation (>90%) the average heat load to the main vessel wall would not exceed 0.15 MW/m².

With this method the long standing problem of excess average heat loads seems to be solved, provided all the other requirements for a burning fusion plasma are also fulfilled:

- acceptable fuel dilution by impurities in the plasma centre
- sufficient energy confinement
- no extreme divertor detached

The optimization of this concept based on improved understanding of the underlying mechanisms is a key issue of ongoing research in present machines and later in particular on ITER.

References

1. D.J. Campbell *Physics of Plasmas* **8** 5 (2001) 2041
2. K.L. BELL et al., "Atomic and molecular data for Fusion," *UKAEA-Report CLM-R216* (1982)

3. I.I. SOBELMAN et al., "Excitation of Atoms and Broadening of Spectral Lines," *Springer Verlag Berlin* (1981)
4. R.V. JENSEN AND D.E. POST, *Atomic Data and Nuclear Data Tables* Vol. 20, No. 5 (1977)
5. SUMMERS H.P. AND MCWHIRTER R.W.P. in "Applied Atomic Collision Physics", Vol.2, eds. Massey et al. *Academic Press* London (1984)
6. J. NEUHAUSER et al., *Nucl. Fus.* **26** 12 (1986)
7. H.A. CLAASSEN et al. *J. Nucl. Mater.* 176-177 (1990) 398
8. U. SAMM et al., "Radiation cooling with intrinsic and injected impurities in the plasma boundary of a limiter tokamak," in *Proc. of the 18th Europ. Conf. on Plasma Physics and Controlled Fusion* Berlin (1991) Vol.3, 157
9. M.Z. TOKAR', *Plasma Phys. Contr. Fusion* **36** (1995) 1819
10. M.Z. TOKAR', *Transactions of Fusion Technology* **37** (2000)
11. B. UNTERBERG and U.SAMM, *these lectures*
12. P.C.STANGEBY, *The Plasma Boundary of Magnetic Fusion Devices*, Plasma Physics Series, IOP Publishing Ltd, Bristol UK (2000)
13. U. SAMM et al., "Plasma edge cooling by impurity radiation in a tokamak," *Proc. of the 14th Inter. Conf. on Plasma Physics and Controlled Nucl. Fusion* IAEA, Würzburg (1992)
14. U. SAMM et al., *Plasma Phys. Contr. Fusion* B167-B175 (1993)
15. A.M. MESSIAEN et al. *Phys. Rev. Lett.* 77 (1996) 2487
16. A. KALLENBACH et al., *Nucl. Fusion* **35** 10 (1995) 1231
17. J. NEUHAUSER et al. *Plasma Phys. Contr. Fusion* **37** (1995) A 37
18. U. SAMM et al., "Plasma Edge Physics with Siliconization in TEXTOR", *Journal of Nucl. Mater.* 220-222 (1995) 25
19. U. SAMM et al., "Properties of detached plasmas", *KFA-Report Jül-2123* (1987)
20. B. LIPSCHULTZ, "Review of MARFE-phenomena in tokamaks," *J. Nucl. Mater.* 145-147 (1989) 15
21. G. SERGIENKO et al., "Investigation of MARFEs in TEXTOR," in *Proc. of the 20th Europ. Conf. on Plasma Physics and Controlled Fusion* Lisbon (1993) **II** 667
22. S. KONOSHIMA et al., *J. Nucl. Mater.* **313-316** (2003) 888
23. E.A. LAZARUS et al., "Confinement in Beam-Heated Plasmas: The Effect of Low-Z Impurities", in *Nucl. Fusion* **25** 2 (1985) 135
24. R.R. Weynants et al *Nucl. Fusion* **39** (1999) 1637
25. M.Z. TOKAR', *Phys. Rev. Lett.* 84 (1995) 895
26. J. ONGENA et al., *Physics of Plasmas*, 8 (5) 2188. (2001)
27. D. REITER et al., "Helium removal from tokamaks," *Plasma Physics and Controlled Fusion* Vol.33, No. 13 (1991) 1579